¹ Optimization of 3D controlled ELM-free state with recovered global confinement

- ² for tokamak fusion plasmas
- ³ S.K.Kim,¹ R.Shousha,¹ S.H.Hahn,² A.O.Nelson,¹ J.Wai,¹ S.M.Yang,³ J.-K.Park,³
- ⁴ R.Nazikian,³ Y.M.Jeon,² Y.In,⁴ J.H.Lee,² J.Kim,² C.Y. Lee,⁵ Y.-S. Na,⁵ and
- ₅ E.Kolemen^{1, 3}
- ⁶ ¹⁾Princeton University
- ⁷ ²⁾Korea Institute of Fusion Energy
- ⁸ ³⁾Princeton Plasma Physics Laboratory
- ⁹ ⁴⁾Ulsan National Institute of Science Technology
- ¹⁰ ⁵⁾Seoul National University
- 11 (Dated: 14 July 2021)

Mitigation of deleterious heat flux from edge-localized modes (ELMs) on fusion reactors 12 is often attempted with 3D perturbations of the confining magnetic fields. However, the 13 established technique of resonant magnetic perturbations (RMPs) also degrades plasma 14 performance, complicating implementation on future fusion reactors. In this paper, we 15 introduce an adaptive real-time control scheme as a viable approach to simultaneously 16 achieve both ELM-free states and recovered high-confinement ($\beta_N \sim 1.91$, $\beta_p \sim 1.53$, and 17 $H_{98} \sim 0.9$), demonstrating successful handling of a volatile complex system through adap-18 tive measures. We show that, by exploiting a salient hysteresis process to adaptively min-19 imize the RMP strength, stable ELM suppression can be achieved while actively encour-20 aging confinement recovery. This is made possible by a self-organized transport response 21 in the plasma edge which reinforces the confinement improvement through a widening of 22 the ion pedestal and promotes control stability, in contrast to the deteriorating effect on 23 performance observed in standard RMP experiments. These results establish the real-time 24 approach as an up-and-coming solution towards an optimized ELM-free state, which is 25 an important step for the operation of ITER and reactor-grade tokamak plasmas. Notably, 26 the real-time adaptive control scheme introduced here provides a path towards economic 27 fusion reactors by maximizing the fusion gain while minimizing damage to machine com-28 ponents. 29

For any fusion energy source to be viable in the global marketplace, it must be able to produce 30 large amounts of electricity without incurring significant damage on the machine. The leading ap-31 proach towards this goal is a tokamak run robustly in the high confinement mode (H-mode), which 32 is characterized by a narrow edge transport barrier responsible for significantly elevated plasma 33 pressures throughout the device¹. This "pedestal" not only enhances performance in the core re-34 gion but also increases the non-inductive current, improving the fusion economy by reducing the 35 external heating and recirculating power required for steady-state operation. Because of these ad-36 vantages, the ITER baseline scenario² plans to utilize H-mode plasmas to demonstrate ignition in 37 a tokamak for the first time. However, H-mode also presents serious risks to reactor operation, 38 most prominently through the creation of dangerous edge instabilities called edge localized modes 39 (ELMs)³. These rapid relaxations of the pedestal density and temperature result in intense tran-40 sient heat fluxes on the reactor walls, leading to undesired material erosion and surface melting 41 which will not be acceptable in a reactor scenario^{4,5}. Therefore, to retain the tokamak design as a 42 viable option for fusion reactors, it is critical that we develop methods to routinely suppress ELM 43 events without degrading the plasma performance. 44

One of the most effective methods to control ELMs is to apply resonant magnetic perturba-45 tions (RMPs) using 3D coils⁶⁻⁹. RMPs suppress ELMs by causing additional transport¹⁰⁻²³ in 46 the pedestal, degrading its height to a point where ELMs are no longer unstable²⁴⁻²⁶. However, 47 this inevitably comes at the considerable expense of global confinement deterioration, decreased 48 access to high-performance plasma regimes and thus depleted economic prospects. This degrada-49 tion tends to be greater with a lower toroidal wave number (n) of RMP. Even so, the use of low-n50 configurations will be important at the reactor level due to the strong decay of external fields in the 51 thick shielding between the plasma and field coils. Undoubtedly, the compatibility of RMP ELM 52 suppression with high confinement operation requires urgent exploration. 53

In this context, we report on an adaptive RMP scheme capable of maximizing plasma perfor-54 mance while maintaining robust ELM suppression. With this new technique, up to $\sim 70\%$ of the 55 RMP-induced performance degradation can be quickly recovered, returning the plasma to a high-56 power state suitable for future reactors. By exploiting a salient hysteresis process on the KSTAR 57 tokamak²⁷, we find that RMP-induced transport does not just produce a negative influence on 58 confinement (as is typically assumed) but instead also opens up a pathway to strong recovery of 50 plasma performance that is accessible to a highly-optimized controller. This leads to the concur-60 rent establishment of high confinement plasmas and sustained ELM suppression at normalized 61

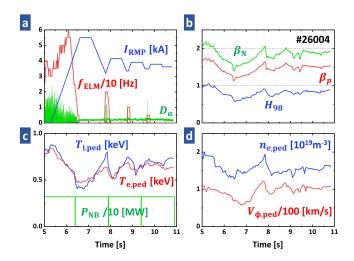


FIG. 1. Plasma parameters for an ELM suppression discharge (#26004) with adaptive RMP control. a RMP coil current (blue), D_{α} emission (green) near outer divertor target, and detected ELM frequency (red). b H_{98} (blue), β_N (green), and β_p (red). c Pedestal height of ion (red), electron (blue) temperature, and NBI heating power (green). d Pedestal height of electron density (blue) and toroidal rotation of carbon (6+) impurity (red).

performance close to the ITER-baseline level, reaching $\beta_N \sim 1.91$, $\beta_p \sim 1.53$, and $H_{98} \sim 0.9$. 62 Here, $\beta_{\rm N} = \frac{aB_T}{I_p} \frac{p}{B^2/2\mu_0}$ is the normalized beta, $\beta_{\rm p} = \frac{p}{B_p^2/2\mu_0}$ is the poloidal beta, and $H_{98} = \tau_{\rm exp}/\tau_{98}$ 63 is the thermal energy confinement quality compared to the standard H-mode plasmas, where p is 64 the averaged plasma pressure, a is the minor radius, I_p is the total plasma, B_T is the toroidal mag-65 netic field, B_p is the poloidal magnetic field, B is the total magnetic field, τ_{exp} is the experimental 66 thermal energy confinement time, and τ_{98} is the empirically derived confinement time using stan-67 dard H-mode database²⁸. Since H_{98} enters to the power of 3.23 in determining the fusion gain 68 $Q_{\rm fus}^{29}$, where $Q_{\rm fus}$ is the ratio between produced fusion energy over input, the strong recovery of 69 H_{98} demonstrated in this work allows a substantial reduction of fusion cost, establishing a means 70 with which RMPs can be used for ELM suppression to enable commercial-grade fusion devices. 71

72 I. RESULTS

Optimized pedestal using adaptive control. Figure.1 shows an example of H-mode plasma with fully suppressed ELMs via adaptive feedback RMP amplitude control. In this discharge, a hysteresis effect is utilized where ELM suppression can be maintained over long periods with a lower RMP strength than initially required for access to the ELM suppression regime¹⁷. Because

reduction of the RMP amplitude leads to an increased pressure pedestal height, this enables global 77 confinement recovery in an ELM-free state³⁰ by adjusting RMP levels. To avoid ELMs while 78 maximizing the confinement, we use a preset low n = 1 RMP waveform⁸ and apply real-time 79 feedback to control its amplitude. During the plasma current flattop before applying RMP, with 80 $I_p=0.51$ MA and ~ 3 MW of neutral beam injection heating, $\beta_{
m N}\sim 2.13$, $\beta_{
m p}\sim 1.71$, and $H_{98}\sim$ 81 1.03, close to the targets of the proposed ITER baseline scenario. In this discharge, the plasma 82 edge safety factor $q_{95} \sim 5$, which is higher than the target value of $q_{95} \sim 3$. Here, q_{95} is defined 83 as the pitch of the magnetic field line in the edge where the normalized poloidal flux (ψ_N) is 84 95%. However, after achieving the first stable ELM suppression through traditional means (7.1 85 s), the plasma performance significantly decreases to $\beta_{\rm N} \sim 1.62$, $\beta_{\rm p} \sim 1.30$, and $H_{98} \sim 0.68$. The 86 30% reduction in overall confinement by RMP mainly comes from degradation in density and 87 temperature pedestal, as shown in Fig.1c, d. Such extensive confinement and H_{98} degradation is 88 a well-known general trend in low-n RMP experiments³¹⁻³³ and will not be acceptable in a future 89 fusion reactor because this leads to a significant increase in fusion cost. 90

After this initial degradation, the real-time adaptive ELM control scheme starts to recover the 91 original performance before RMPs were introduced while maintaining stable ELM suppression. 92 The controller leverages the D_{α} emission signal near the outer divertor target to calculate the 93 frequency of ELMs $(f_{ELM})^{34}$. To achieve ELM suppression, the RMP amplitude (or coil cur-94 rent, $I_{\rm RMP}$) is raised until $f_{\rm ELM}$ decreases to 0, i.e., ELM suppression. Then, during the resulting 95 ELM-free period, the controller lowers the RMP strength to raise the pedestal height until ELMs 96 reappear, at which point the control again starts to ramp up the RMP amplitude until suppression 97 is recovered (Fig.1a). In the experiment presented in Fig.1, there are 0.5 s of RMP flattop intervals 98 between the RMP-ramp up and down phase to achieve saturated RMP response. Throughout this 99 process, we adjust the lower bound of $I_{\rm RMP}$ to match the value where the most recent ELM returns. 100 This adaptive constraint reduces the likelihood of ELM suppression loss and control oscillation. 101 The feedback system leads the plasma to a converged operating point that optimizes both ELM-102 free operation and confinement, recovering most of the performance lost in the initial application 103 of RMP. 104

In the selected discharge, this adaptive ELM control scheme achieves a stable ELM-free phase at 10.5 s with improved global confinement, as shown in Fig.1b. Although a few ELMs occur before convergence, the controller successfully reaches a stable operating point with minimized ELMy periods. In the final state, the plasma performance shows $\beta_N \sim 1.91$, $\beta_p \sim 1.53$, and

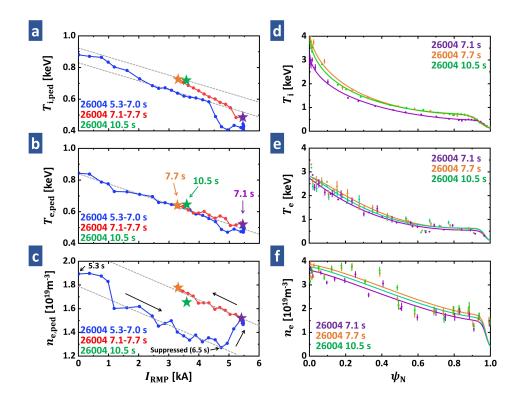


FIG. 2. Pedestal height (left) and core plasma profiles (right) for RMP ramp-up (5.3-7.1 s, blue), down (7.1-7.7 s, red), first saturated ELM-suppression (7.1 s, purple), first optimized suppression (7.7 s, or-ange), and finally optimized suppression (10.5 s, green). a-c Pedestal height of ion, electron temperature and electron density. d-f Core ion, electron temperature, and electron density with statistical error bars. Ion temperature is measured by a charge-exchange recombination system for carbon (6+) impurities. Electron temperature is measured by the Thomson Scattering and Electron cyclotron emission system. Electron density is measured by the Thomson Scattering and Two-color interferometry system.

 $H_{98} \sim 0.9$, recovering up to 68% of the original confinement degradation. Such increase in H_{98} 109 is especially important as this leads to the 60% recovery in $Q_{\rm fus}$ degradation, thus emphasizing 110 the performance of adaptive control. The enhanced confinement quality occurs with the recov-111 ery of both the temperature and density pedestals. As can be seen in Fig.1c, d, all pedestals are 112 significantly improved from the first ELM suppression phase. For example, electron $(T_{e,ped})$ and 113 ion ($T_{i,ped}$) temperature pedestals increase by 22% and 50%, respectively. In addition, the electron 114 density pedestal ($n_{e,ped}$) is also recovered by 10% at the same time. Interestingly, $H_{98} \sim 0.9$ at 115 10.5 s is much larger than $H_{98} \sim 0.75$ at 6.2 s, even with the same $I_{\text{RMP}} = 3.6$ kA. This indicates 116 that the confinement recovery by adaptive approach is not solely attributable to decreased $I_{\rm RMP}$, 117 but rather that another contributor leads the plasma to a reinforced high-confinement state. 118

We note that the ion temperature pedestal exhibits significant recovery compared to the other 119 channels. This is mainly due to the rapid and significant increase of ion pedestal height by 120 decreasing RMP strength. The traces of pedestal height versus $I_{\rm RMP}$ before the first ELM reap-121 pearance (5.3-7.7s) reveal this trend, as shown in Fig.2a-c. $n_{e,ped}$ and $T_{e,ped}$ have a similar depen-122 dence on I_{RMP} during the pedestal degradation (5.3-6.5s) and recovery (7.1-7.7s) phases, showing 123 $\frac{\Delta n_{\rm e,ped}}{\Delta I_{\rm RMP}} \sim -10^{15}/m^3$ A and $\frac{\Delta T_{\rm e,ped}}{\Delta I_{\rm RMP}} \sim -0.06$ eV/A. However, $T_{\rm i,ped}$ in the recovery phase shows a 124 50% larger response of -0.09eV/A compared to the degradation phase, -0.06eV/A. The differ-125 ence of responses in these phases leads to the faster and larger recovery of the ion pedestal. As 126 shown in Fig.2d-f, all radial profiles in the core plasma are almost identical during the recovery 127 phase. Therefore, the improved confinement by decreasing RMP strength results from increased 128 $n_{\rm e,ped}$, $T_{\rm e,ped}$, and $T_{\rm i,ped}$, with the last one dominant. In particular, ~ 67% of improvement comes 129 from the ion pedestal, and this is responsible for reinforced recovery by adaptive control. The 130 large growth of $T_{i,ped}$ is mainly due to the simultaneously increased upper limit of $T_{i,ped}$ before the 131 loss of ELM suppression and its enhanced response to the RMP strength. In addition, ne,ped shows 132 a large increase near $I_{\rm RMP} \sim 5$ kA (Fig.2c), which can be attributed to reduced particle pumping 133 from ELMs. This occurs before 7 s and does not directly contribute to confinement recovery 134 beginning at 7.1 s. However, it still strengthens the confinement recovery with increasing $T_{i,ped}$. 135 136

Advantages of the adaptive ELM control for achieving safe ELM suppression. In standard 137 H-mode discharges, strong RMPs are favorable for entering the ELM suppression but also raises 138 the possibility of dangerous plasma destabilization. Too large of an RMP field in the core plasma 139 normally leads to a locking of plasma rotation and invokes a disastrous core instability called a dis-140 ruption, as seen in Fig.3a. Core locking (or disruptions) terminate the plasma and forms transient 141 heat fluxes on the tokamak walls which are even more severe than ELMs. Unfortunately, plasma 142 disruption is easier with low-n RMPs. Therefore it is vital to maintain the RMP strength between 143 the thresholds of ELM suppression and disruption. To complicate this process, these thresholds 144 change in time with various plasma parameters and are often hard to theoretically predict. The 145 database³³ for n = 1 RMP ELM suppression in KSTAR reveals broadly scattered experimental 146 thresholds showing $1 \sim 2$ kA variations, and empirical prediction is also challenging due to their 147 sensitivity to plasma parameters. For these reasons, in the present experiments, a series of dis-148 charges are used to find safe RMP strength for ELM suppression. This approach will not be 149 applicable in a fusion reactor, where a single disruption can result in the termination of machine 150

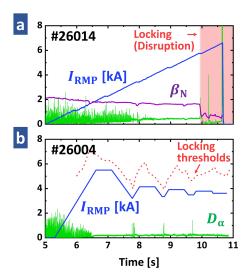


FIG. 3. Plasma parameters for a RMP-induced disruption and suppression discharge with n = 1 RMP in KSTAR. a RMP coil current (blue), D_{α} emission (green), and β_{N} (purple) of discharge #26014. Onset of locking (disruption) is marked as a red region. b RMP coil current (blue) and D_{α} emission (green) of discharge #26004. The disruption thresholds in I_{RMP} is marked as a red dotted line.

151 life.

Notably, the adaptive approach lowers the RMP strength after entering the ELM-free state and 152 maintains it near the levels for marginally stable ELM suppression. This automatically avoids 153 touching the disruptive limits. As shown in Fig.3b, the RMP strength stays safely below the dis-154 ruption threshold throughout the example discharge, highlighting the advantages of this adaptive 155 scheme for achieving stable ELM suppression. Here, the disruption thresholds are predicted from 156 adjacent RMP-disruption experiments and ideal RMP response calculations. Although adaptive 157 RMP control will be ineffective if only a small margin exists between the thresholds for suppres-158 sion and disruption, it still reduces the necessity of extensive optimization of the RMP geometry 159 for locking avoidance, which often comes at the expense of other important parameters or opera-160 tional degrees of freedom. 161

162

Improved ELM stability and ion pedestal response by RMP-induced transports. Instead of causing only degradation of pedestal, RMP-induced pedestal transport facilitates the improvement of the $T_{i,ped}$ limit and its response to the RMP strength by broadening the ion-pedestal. RMPinduced transport on the ion pedestal can be found from the analysis of the ion pedestal profiles in detail. Fig.4a, b illustrate ion pedestal and $E \times B$ flow profiles for five times between 5.3 and 7.7

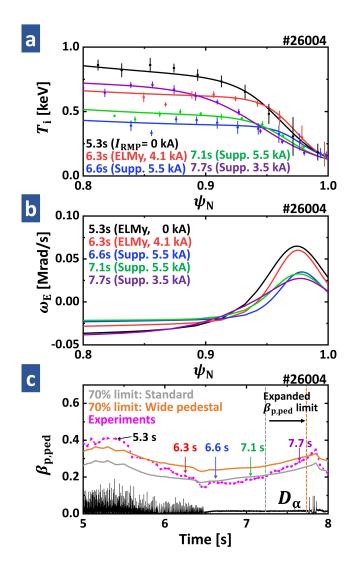


FIG. 4. Time traces of pedestal profiles and stability limits during adaptive ELM control (#26004). a Ion pedestal profiles with statistical error bars are shown for five different time slices. **b** ExB flow profiles $(\omega_{\rm E})$ at pedestal are shown for five different time slices. **c** 70% of ELM stability limit for $\beta_{\rm p,ped}$ with (orange) and without (gray) wide ion pedestal, calculated from EPED code. Experimentally measured $\beta_{\rm p,ped}$ (magenta) and D_{α} emission (black) are also shown. The dotted lines show $\beta_{\rm p,ped}$ limits during ELM-free state imposed by pedestal stability with (gray) and without (orange) wide ion pedestal.

s. Before ELM suppression (5.3-6.3 s), $T_{i,ped}$ decreases with I_{RMP} , while the pedestal gradient is well sustained (or even slightly increased). After ELM suppression (> 6.5 s), however, the pedestal stiffness starts to change. The transition from 6.6 to 7.1 s shows broadening of the ion pedestal and decreasing of its gradient. This widening is maintained in the pedestal recovery phase up to 7.7 s. The decrease in pedestal height and gradient are both due to RMP-induced transport. However, the rapid broadening of the ion pedestal after ELM suppression indicates that its gradient is not governed by the transport affecting the pedestal height but instead by an "additional" transport source that occurs in the ELM suppression phase.

The change in ion pedestal width improves the ELM stability. In theory, pedestal pressure 176 (P_{ped}) or pedestal poloidal beta $(\beta_{p,\text{ped}} = \frac{P_{\text{ped}}}{B_p^2/2\mu_0})$ should stay under the stability limit to avoid 177 the reappearance of ELM crashes. Stability analysis confirms that $\beta_{p,ped}$ stays below 70% of the 178 stability limit during the ELM suppression phase. This stability limit is known to improve with 179 increased pedestal width³⁵. Therefore, widened pressure pedestal via ion-pedestal broadening 180 allows for higher $\beta_{p,ped}$ during the ELM-free phase. Numerical analysis reveals that the $\beta_{p,ped}$ 181 limit increases by 53% due to ion pedestal broadening. This change is presented in Fig.4c. With 182 the expansion of the $\beta_{p,ped}$ limit illustrated as dotted lines, $\beta_{p,ped}$ can further increase from 0.2 183 (gray dotted line) to 0.31 (orange dotted line). This enhanced $\beta_{p,ped}$ limit allows access to higher 184 $T_{i,ped}$ in the ELM suppression phase. 185

The broader ion-pedestal also can lead a larger response of $T_{i,ped}$ on RMP strength. Inspired from (Hu et al. 2020)³⁶, the change of pedestal height (ΔT_{ped}) by ΔI_{RMP} can be described as Eq.1,

$$\frac{\Delta T_{\text{ped}}}{\Delta I_{\text{RMP}}} \approx \nabla T_{\text{ped}} \sum_{m \ge q_{\text{ped}}} \frac{\partial W_{\text{m,n}}}{\partial I_{\text{RMP}}},\tag{1}$$

where $W_{m,n}$ and ∇T_{ped} are the (m,n) island width and pedestal gradient, respectively. q_{ped} is an 188 edge safety factor on the pedestal top. This expression is based on the concept where ΔT_{ped} is 189 the accumulation of profile flattening by the islands in the pedestal region. We note that constant 190 ∇T_{ped} over the pedestal region is assumed to make interpretation easier. This expression addresses 191 that pedestal height changes more rapidly with RMP strength as the pedestal gradient grows and 192 q_{ped} decreases. With the given q profile monotonic, q_{ped} is reduced by increasing pedestal width. 193 The largely broadened ion pedestal can lead to a stronger response of $T_{i,ped}$ despite the decrease of 194 ion pedestal gradient. In addition, ion pedestal is known to be heavily influenced by neoclassical 195 transport^{15,37,38}. Here, neoclassical heat flux by RMPs is roughly proportional to $I_{\rm RMP}^2$, and it 196 increases more rapidly with the smaller radial electric field and its gradient^{39,40}. Because a wide 197 ion pedestal reduces the electric field^{19,41} at the pedestal (Fig.4b), this correlation also contributes 198 to improving the response of $T_{i,ped}$. 199

On the other hand, the responses of $n_{e,ped}$ and $T_{e,ped}$ to RMP strength are almost identical whether or not the ELMs are fully suppressed. This means that additional RMP-induced transport in the ELM-free phase has a smaller effect on the electron density and temperature pedestal gra-

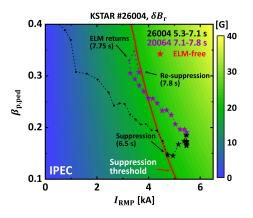


FIG. 5. The pressure pedestal height versus RMP strength during adaptive ELM control (#26004). The time traces of $\beta_{p,ped}$ in #26004 discharge for 5.3-7.1 s (black) and 7.1-7.8 s (purple) with varying I_{RMP} . ELM-free states are marked as star dots. Contours of δB_r at pedestal region from ideal response calculation using IPEC are also shown. Experimentally derived δB_r threshold for ELM suppression is drawn as a red curve.

dient. Although the electron pedestal width has considerable uncertainty due to limitations in the resolution of edge diagnostics, its value lies between 4-6% in normalized poloidal flux without showing a considerable widening like ion pedestal, suggesting that additional transport has only a relatively small effect on electron channels.

207

Advantages of RMP-induced transport and wide ion pedestal in adaptive ELM control. In-208 creased $T_{i,ped}$ response by RMP-induced transport leads to an extensive recovery of $T_{i,ped}$ during 209 RMP ramp-down and makes an ion pedestal higher than the RMP ramp-up phase (ELMy) even 210 with the same RMP strength. In addition, enhanced pedestal stability allows for larger $T_{i,ped}$ before 211 the return of ELMs. The synergy between these effects boosts the pedestal recovery and enables 212 adaptive control to maximize the confinement, resulting in a much higher pedestal than during the 213 initial phase of ELM suppression, as shown in Fig.5, which illustrates $\beta_{p,ped}$ versus I_{RMP} . The 214 changes to the pedestal from 5.3 to 7.8 s are shown, and the ELM suppressed states are marked 215 with star points. 216

Another advantage of RMP-induced transport is that it improves the control stability. Adaptive control can be unstable due to a bifurcation of the plasma state during transitions between ELMy and ELM-free regimes, which causes oscillation of the control system. In particular, it can take a long time or even become impossible for a controller to find the optimal solution because of

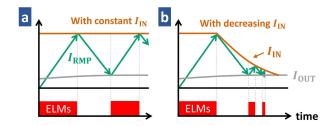


FIG. 6. Schematic diagram of adaptive ELM control using RMPs. Here, RMP threshold for ELM suppression entry (I_{IN} , orange) and exit (I_{OUT} , gray) are drawn. Time trance of I_{RMP} (green) and onset of ELMs (red box) are also shown. Expected time trace of adaptive ELM control with **a** constant I_{IN} and **b** decreasing I_{IN} in time.

the sudden jump in RMP strength required for re-entry (I_{IN}) to or exit (I_{OUT}) from ELM suppression. The schematic diagram in Fig.6a illustrates how this characteristic will delay the control convergence. In practice, ELM control must be done quickly to minimize damage to the reactor, so an adaptive approach is generally hard to use in such a bifurcating system. However, RMPinduced transport eases these control difficulties by reducing I_{IN} during adaptive control, as shown in Fig.6b.

It has been shown that the plasma enters the ELM suppression state above a certain $\delta B_{\rm r}$ 227 threshold⁴², where $\delta B_{\rm r}$ is the perturbed radial field strength at the pedestal. This threshold (~ 20 228 G) for the reference discharge is shown as the red contour of Fig.5. Here, $\beta_{p,ped}$ amplifies the per-229 turbed field⁴², and the same δB_r can be obtained with a smaller $I_{\rm RMP}$ with larger $\beta_{\rm p,ped}$. Because 230 RMP-induced transport enhances $\beta_{p,ped}$ in an ELM-free state, this leads to a lower I_{IN} , making 231 access to the next ELM suppression regime easier. The ELM suppression of 7.8 s shown in Fig.5 232 results from reduced I_{IN} compared to the former one at 6.5 s. Thus, I_{IN} for each suppression 233 entry changes as $4.9 \rightarrow 3.6 \rightarrow 3.53 \rightarrow 3.5$ kA, as seen in Fig.1(a), resulting in fast and stable sys-234 tem optimization. This interesting example shows uncommon positive effect^{43,44} of self-organized 235 transport on pedestal confinement. 236

²³⁷ We note that such an RMP-induced hysteresis shown in Fig5 is not trivial to be produced in ²³⁸ the experiment as it conventionally requires a delicate pre-programmed RMP waveform. This ²³⁹ leads to difficulties in investigating and exploiting the hysteresis, which is critical to optimize ²⁴⁰ the ELM-free state. In this respect, adaptive RMP control is an effective methodology as it can ²⁴¹ automatically generate the hysteresis and utilize it. In addition, the adaptive scheme has been suc-²⁴² cessfully operated for more than a hundred confinement times (~ 5 s) of KSTAR, and therefore, this control is also expected to be applicable to long pulse plasma in ITER.

244

The origin of broadened ion-pedestal. It is worth pointing out that successful adaptive con-245 trol in these experiments is mainly due to a broadened ion pedestal during the ELM suppression 246 phase. As shown in Fig.7a, the ion heat diffusivity (χ_i) of the pedestal region rapidly increases 247 via additional transport after transitions to the ELM-free state. In addition, the pedestal heat dif-248 fusivity does not change much during 7.1-7.7 s, indicating that it is insensitive to the decreasing 249 $I_{\rm RMP}$. It has been reported that the neoclassical transport effect dominates ion heat transport under 250 RMPs^{37,38}. However, this collisional transport strongly depends on the RMP strength. There-251 fore, the broadened ion pedestal does not seem to be related to the neoclassical process. Here, 252 χ_i at $\psi_N = 0.96$ exceeds neoclassical level ($\geq 0.4m^2/s$) in all cases, supporting the existence of 253 additional transport. 254

Fluctuation measurements on KSTAR ($k_y \rho_s < 0.1$) reveal significant edge turbulence triggered 255 by RMPs^{25,26,45} after ELM suppression, where k_y is the bi-normal wave number, $\rho_s = \sqrt{2m_iT_e}/eB$ 256 is the hybrid Larmor radius, and m_i is deuterium mss. Fig.7c, d illustrate the spectrogram and the 257 coherence strength of δT_e and δn_e fluctuations at $\psi_N \sim 0.96$. Fig.7e shows the poloidal magnetic 258 field fluctuations (δB_{pol}) at the inner wall. Here, δT_e and δn_e have strong coherence over the 259 frequency range of 20-100 kHz. The magnetic fluctuations in the 80-400 kHz range are also 260 observed during the same period. As shown in Fig.7f, they show an immediate instigation of 261 turbulence as ELM suppression begins followed by quick saturation within 200 ms. We note that 262 coherence before 6.4 s comes from ELM noise, and a magnetic signal of <50 kHz is responsible 263 for core modes. It is noteworthy that the strength of coherent fluctuations remains almost identical 264 during 7.1-7.7 s. Here, the widening of the ion pedestal coincides with the occurrence of edge 265 fluctuations. Furthermore, they are both insensitive to RMP strength. Therefore, these similarities 266 support the claim that the ion pedestal is widened primarily due to increased heat diffusivity by 267 edge turbulence. 268

Linear gyrokinetic simulations confirms that enhanced edge turbulence may occur in the ELM suppression phase. As shown in Fig.7b, the linear growth rates (γ/γ_E) of turbulence mode exceed the onset limit (>1) after the transitions to the ELM-free state. This is mainly due to decreased stabilizing effect from the ExB shearing rate (γ_E)^{46,47}, which comes from the degraded pressure pedestal (Fig.4b). It turns out that the excited modes are correlated with the ITG/TEM hybrid. Here, the bi-normal wave length $k_y \rho_s \sim 0.3$ and real frequency ~ 51 kHz of the most unstable

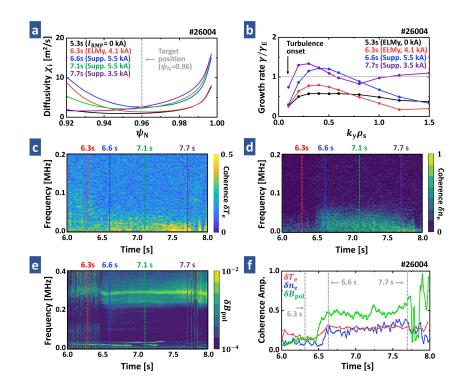


FIG. 7. A broadened ion temperature pedestal by RMP-induced transport during ELM-suppression state. a Radial profiles of ion heat diffusivity (χ_i) for five different time slices. b The growth rates of instability calculated from Gyro-kinetic simulation code CGYRO. c Coherence of edge T_e fluctuation from Electron cyclotron emission imaging system. d Coherence of edge n_e fluctuation from Beam emission imaging system. e Measured δB_{pol} fluctuation at inner wall from Mirnov coil. f Time trace of normalized integrated coherence amplitude of T_e (red), n_e (blue), and B_{pol} (green) fluctuations over the frequency space.

mode exhibits similar properties to the measured fluctuations of electron channels. The simulation 275 results show that ion thermal diffusion can be increased with these unstable modes, supporting the 276 idea of ion pedestal broadening by turbulence. However, theoretical analysis on RMP-induced tur-277 bulence still has many missing pieces. Recent studies have shown that the characteristics of trans-278 port in the presence of RMP deviates significantly from linear gyrokinetic calculations, raising 279 the importance of non-linearity⁴⁸ and non-locality⁴⁹. In the future, nonlinear gyrokinetic studies 280 including these effects will shed further light on the accurate description of edge turbulence under 281 RMPs. 282

The considerable effect of RMP-induced transport on ion heat diffusion might inconsistent with the general trend of other devices^{16,17,32}, where such turbulence mainly affects electron channel and has a minor effect on ion transport. Although it is difficult to evaluate the turbulence effect

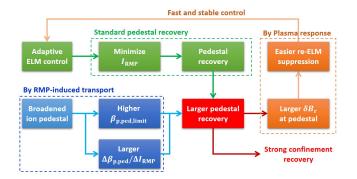


FIG. 8. Schematic diagram of correlation between adaptive ELM control and pedestal recovery. Here, it is noteworthy that the strong recovery of confinement is also attributable to the widened ion pedestal by RMP-induced transport during ELM suppression phase.

on n_e and T_e due to limitations in the diagnostics, we still confirm that there is a clear correlation between edge fluctuation and ion pedestal. Therefore, this observation suggests new possible role of turbulence on the ion pedestal under the low-n(= 1) RMP and ELM-free states.

289 II. DISCUSSION

We have achieved successful optimization of a controlled ELM-free state with highly recov-290 ered confinement by ~ 60%, maintaining $\beta_N \sim 1.91$, $\beta_p \sim 1.53$, and $H_{98} \sim 0.9$, with the original 291 degradation in fusion gain largely recovered. This novel adaptive approach exhibits compatibility 292 between RMP ELM suppression and high confinement. In addition, it provides a reliable strategy 293 to achieve stable ELM-free access by preventing RMP-induced disruption. It is noteworthy that 294 the remarkable recovery of confinement is not solely attributable to adaptive RMP control but also 295 to a widened ion pedestal resulting from RMP-induced transport that promotes pedestal recovery 296 by improving the ion response and ELM stability and facilitates fast, stable, and reinforced control 297 optimization (Fig.8). This feature, which can be correlated to the turbulent process, is a good ex-298 ample of a system that transitions to an optimal state through a self-organized response to adaptive 290 modulation. These results with low n = 1 RMP confirm that adaptive ELM control is a highly 300 promising approach towards optimizing the ELM-free state, potentially solving one of the most 301 challenging obstacles for viable and economical fusion energy. 302

However, there are remaining features to be improved for a "complete" adaptive ELM control picture. As shown in Fig.1a, the current approach is based on ELM detection and thereby inevitably faces several ELMs during control. This limitation could be critical at the reactor level, where a single ELM can already be dangerous. Thus, a way to detect the loss of ELM suppression in advance of the ELM re-occurrence is needed. Here, the behavior of edge turbulence suggests the potential solution. The amplitude of magnetic fluctuation during the ELM-free phase shows a rapid decrease 70 ms before the return of ELMs at 7.75 s (Fig.7f). Such an abrupt change in magnetic signals is an effective indicator of suppression loss. Therefore, this property can be utilized in real-time to entirely avoid the return of ELM to achieve truly ELM-free optimization.

Previous work has shown that the effectiveness of RMP ELM suppression can be enhanced by physics model-based 3D geometric optimization⁵⁰. Since this adaptive ELM control scheme maximizes the plasma performance for a given scenario, any additional improvements from external forces will be augmented by the adaptive scheme. This makes the adaptive approach a prime candidate to fully exploit existing physics models for RMP ELM suppression. Future integration of these features will lead to broader operational freedom and higher confinement recovery, as well as the development of advanced ELM control techniques for ITER and future tokamaks.

319 III. METHODS

KSTAR tokamak. The KSTAR tokamak is the largest magnetic fusion devices in Republic of Korea, supported by the Korea Institute of Fusion Energy (KFE) and the Government funds. It has the plasma major radius $R_0 = 1.8$ m, minor radius $a_0 = 0.45$ m, and the toroidal magnetic field $B_T = 1.8 - 2.3$ T at major radius R_0 . The n = 1 RMP ELM suppression discharge on KSTAR can be reproduced at a lower electron density regime (.e.g., Greenwald density fraction ~ 0.4) with a plasma shape having elongation $\kappa \sim 1.71$, upper triangularity $\delta_{up} \sim 0.37$, and upper triangularity $\delta_{low} \sim 0.85$.

327

Radial profile reconstruction. Core ion temperature is measured by charge exchange recombination system⁵¹ for Carbon (6+) impurities at outboard mid-plane. Core electron temperature is measured by the Thomson Scattering⁵² and Electron Cyclotron emission⁵³ system. Core electron density is measured by the Thomson Scattering and Two-color interferometry system⁵⁴. To obtain well-resolved profiles, the data are averaged over 100 ms. The pedestal height is obtained from hyperbolic tangent fits with edge profiles. The equilibria from EFIT code⁵⁵ is used for the radial profile mapping and fitting. 335

Kinetic equilibria reconstruction. Kinetic equilibria are reconstructed for the plasma stability analysis. This equilibrium is calculated from the magnetic reconstruction using EFIT code with the pressure profile (summation of thermal pressure profile from radial profile reconstruction and fast ion pressure from NUBEAM code⁵⁶) and current density profile (core current from motional Stark effect diagnostics⁵⁷ and edge current using NUBEAM, Ohmic and Sauter current models⁵⁸) as a constraint. An iteration scheme is employed to update the thermal profiles, NUBEAM results, and edge current calculation with new kinetic equilibrium.

343

Pedestal stability calculation. The pedestal stability (or ELM stability) limit is predicted using the EPED1⁵⁹ algorithm. The fixed-boundary equilibrium code, CHEASE⁶⁰, is used for accurate equilibrium mapping, and the ideal MHD stability code, MISHKA1⁶¹, is employed for ideal peeling-ballooning³ stability calculation. The linear initial value solver is used to calculate the most unstable mode. All other required parameters are taken from the reconstructed radial profiles and plasma equilibrium.

350

Ideal plasma response calculation. The perturbed radial fields (δB_r) from an ideal plasma response by RMP are calculated using IPEC code⁶² and given magnetic equilibria and I_{RMP} . The core and edge responses are derived through radially averaging δB_r at $\psi_N = 0 - 0.9$ and 0.9 - 1.0, respectively. The thresholds of δB_r for RMP-induced ELM suppression and disruption are obtained from neighboring experiments. The disruption thresholds in I_{RMP} are equivalent to the δB_r thresholds based on the plasma response calculation.

357

Plasma fluctuation measurements. In this work, edge T_e and n_e fluctuations ($k_{\perp}\rho < 1$) are measured from electron emission image spectroscopy (ECEI)⁶³ and beam emission spectroscopy (BES)⁶⁴, respectively. Magnetic field perturbations are captured by the Mirnov coil signal (MC)⁶⁵. The spectrogram of measured fluctuation is derived using Fourier transform. Coherence of electron density and temperature fluctuation is calculated from bi-spectrum analysis with two radially adjacent channels in ECEI and BES, respectively. The ELM peaks and core modes are statistically removed from integrating the amplitude of coherent fluctuations in all channels.

365

³⁶⁶ Gyro-kinetic simulation. The gyrokinetic code, CGYRO⁶⁶, is used in the linear analysis of

micro-instabilities. The linear initial value solver is employed to find the unstable mode in the target radial point with wavelength $k_y \rho_s = 0.1 - 1.5$. This simulation is based on a flux-tube approach with a full gyro-kinetic description for both electron and ion channels. The reconstructed radial profiles and kinetic equilibrium described above are included for the accurate modeling. This calculation is performed at $\psi_N = 0.96$, where the changes of experimental fluctuations are robust.

373

374 IV. ACKNOWLEDGMENTS

The authors would like to thanks KSTAR team. This material was supported by the U.S. Department of Energy, under Awards DE-SC0020372. This research was also supported by R&D Program of "KSTAR Experimental Collaboration and Fusion Plasma Research(EN2021-12)" through the Korea Institute of Fusion Energy(KFE) funded by the Government funds.

379 V. AUTHOR CONTRIBUTIONS

S.K. and R.S. led the experimental demonstration and analysis. R.S. and S.K. develop the adaptive 380 controller. E.K. conceived the original idea of adaptive control. A.O.N. analyzed the micro insta-381 bility with Gyro-kinetic code. S.H., S.Y., J.W. and Y.M. participated in all the experimental pro-382 cedures and support the analysis. R.N. and Y.I. discussed the critical physics picture of ion-scale 383 turbulence and other transports at the pedestal. J.P. and Y.S. discussed the role of RMP response, 384 stability and transport analysis of pedestal region. C.Y conducted the interpretive transport analy-385 sis using ASTRA. J.L. and J.K. analyzed the measured edge fluctuation in electron channel. S.K. 386 wrote the main manuscript text and A.O.N. and all authors reviewed it. 387

388 VI. ADDITIONAL INFORMATION

³⁸⁹ Competing financial interests: The authors declare no competing financial interests.

390 **REFERENCES**

- ¹F. Wagner, G. Fussmann, T. Grave, M. Keilhacker, M. Kornherr, K. Lackner, K. McCormick,
- E. R. Müller, A. Stäbler, G. Becker, K. Bernhardi, U. Ditte, A. Eberhagen, O. Gehre, J. Gern-
- ³⁹³ hardt, G. v. Gierke, E. Glock, O. Gruber, G. Haas, M. Hesse, G. Janeschitz, F. Karger, S. Kissel,
- O. Klüber, G. Lisitano, H. M. Mayer, D. Meisel, V. Mertens, H. Murmann, W. Poschenrieder,
- ³⁹⁵ H. Rapp, H. Röhr, F. Ryter, F. Schneider, G. Siller, P. Smeulders, F. Söldner, E. Speth, K. H.
- Steuer, Z. Szymanski, and O. Vollmer, en"Development of an Edge Transport Barrier at the
 H-Mode Transition of ASDEX," Physical Review Letters 53, 1453–1456 (1984).
- ²A. Sips, J. Schweinzer, T. Luce, S. Wolfe, H. Urano, J. Hobirk, S. Ide, E. Joffrin, C. Kessel,
- S. Kim, P. Lomas, I. Nunes, T. Pütterich, F. Rimini, W. Solomon, J. Stober, F. Turco, P. de Vries,
- JET Contributors, The ASDEX Upgrade team, The DIII-D team, The C-Mod team, The JT-60U
- team, and ITPA-IOS TG members and experts, en "Assessment of the baseline scenario at $q_{.95}$
- ⁴⁰² ~ 3 for ITER," Nuclear Fusion **58**, 126010 (2018).
- ⁴⁰³ ³J. W. Connor, R. J. Hastie, H. R. Wilson, and R. L. Miller, en"Magnetohydrodynamic stability
 ⁴⁰⁴ of tokamak edge plasmas," Physics of Plasmas 5, 2687–2700 (1998).
- ⁴⁰⁵ ⁴A. Loarte, G. Huijsmans, S. Futatani, L. Baylor, T. Evans, D. M. Orlov, O. Schmitz, M. Becoulet,
- P. Cahyna, Y. Gribov, A. Kavin, A. Sashala Naik, D. Campbell, T. Casper, E. Daly, H. Frerichs,
- 407 A. Kischner, R. Laengner, S. Lisgo, R. Pitts, G. Saibene, and A. Wingen, en"Progress on the
- ⁴⁰⁸ application of ELM control schemes to ITER scenarios from the non-active phase to DT opera-⁴⁰⁹ tion," Nuclear Fusion **54**, 033007 (2014).
- ⁴¹⁰ ⁵J. Gunn, S. Carpentier-Chouchana, F. Escourbiac, T. Hirai, S. Panayotis, R. Pitts, Y. Corre,
- R. Dejarnac, M. Firdaouss, M. Kočan, M. Komm, A. Kukushkin, P. Languille, M. Missirlian,
- W. Zhao, and G. Zhong, en"Surface heat loads on the ITER divertor vertical targets," Nuclear

⁴¹³ Fusion **57**, 046025 (2017).

- ⁴¹⁴ ⁶T. E. Evans, R. A. Moyer, P. R. Thomas, J. G. Watkins, T. H. Osborne, J. A. Boedo, E. J. Doyle,
- M. E. Fenstermacher, K. H. Finken, R. J. Groebner, M. Groth, J. H. Harris, R. J. L. Haye, C. J.
- Lasnier, S. Masuzaki, N. Ohyabu, D. G. Pretty, T. L. Rhodes, H. Reimerdes, D. L. Rudakov,
- 417 M. J. Schaffer, G. Wang, and L. Zeng, "Suppression of Large Edge-Localized Modes in High-
- 418 Confinement DIII-D Plasmas with a Stochastic Magnetic Boundary," Physical Review Letters
- **92** (2004), 10.1103/physrevlett.92.235003, publisher: American Physical Society (APS).

- ⁴²⁰ ⁷W. Suttrop, T. Eich, J. C. Fuchs, S. Günter, A. Janzer, A. Herrmann, A. Kallenbach, P. T. Lang,
- T. Lunt, M. Maraschek, R. M. McDermott, A. Mlynek, T. Pütterich, M. Rott, T. Vierle, E. Wol-
- frum, Q. Yu, I. Zammuto, and H. Zohm, en"First Observation of Edge Localized Modes Mit-
- igation with Resonant and Nonresonant Magnetic Perturbations in ASDEX Upgrade," Physical
- 424 Review Letters **106**, 225004 (2011).
- ⁴²⁵ ⁸Y. M. Jeon, J.-K. Park, S. W. Yoon, W. H. Ko, S. G. Lee, K. D. Lee, G. S. Yun, Y. U. Nam, W. C.
- ⁴²⁶ Kim, J.-G. Kwak, K. S. Lee, H. K. Kim, and H. L. Yang, "Suppression of Edge Localized Modes
- ⁴²⁷ in High-Confinement KSTAR Plasmas by Nonaxisymmetric Magnetic Perturbations," Physical
- Review Letters **109** (2012), 10.1103/physrevlett.109.035004, publisher: American Physical Society (APS).
- ⁴³⁰ ⁹Y. Sun, Y. Liang, Y. \. e. Liu, S. Gu, X. Yang, W. Guo, T. Shi, M. Jia, L. Wang, B. Lyu, C. Zhou,
- A. Liu, Q. Zang, H. Liu, N. Chu, H. \. e. Wang, T. Zhang, J. Qian, L. Xu, K. He, D. Chen,
- B. Shen, X. Gong, X. Ji, S. Wang, M. Qi, Y. Song, Q. Yuan, Z. Sheng, G. Gao, P. Fu, and
- B. Wan, "Nonlinear Transition from Mitigation to Suppression of the Edge Localized Mode
 with Resonant Magnetic Perturbations in the EAST Tokamak," Physical Review Letters 117
- (2016), 10.1103/physrevlett.117.115001, publisher: American Physical Society (APS).
- ⁴³⁶ ¹⁰M. E. Fenstermacher, T. E. Evans, T. H. Osborne, M. J. Schaffer, M. P. Aldan, J. S. deGrassie,
 ⁴³⁷ P. Gohil, I. Joseph, R. A. Moyer, P. B. Snyder, R. J. Groebner, M. Jakubowski, A. W. Leonard,
- P. Gohil, I. Joseph, R. A. Moyer, P. B. Snyder, R. J. Groebner, M. Jakubowski, A. W. Leonard,
- 438 O. Schmitz, and the DIII-D Team, en"Effect of island overlap on edge localized mode suppres-
- sion by resonant magnetic perturbations in DIII-D," Physics of Plasmas 15, 056122 (2008).
- ¹¹F. L. Waelbroeck, I. Joseph, E. Nardon, M. Bécoulet, and R. Fitzpatrick, "Role of singular layers
 in the plasma response to resonant magnetic perturbations," Nuclear Fusion 52, 074004 (2012),
 publisher: IOP Publishing.
- ¹²Q. M. Hu, R. Nazikian, B. A. Grierson, N. C. Logan, J.-K. Park, C. Paz-Soldan, and Q. Yu, "The
 density dependence of edge-localized-mode suppression and pump-out by resonant magnetic
 perturbations in the DIII-D tokamak," Physics of Plasmas 26, 120702 (2019), publisher: AIP
 Publishing.
- ¹³R. Fitzpatrick, "Theory of edge localized mode suppression by static resonant magnetic perturbations in the DIII-D tokamak," Physics of Plasmas 27, 042506 (2020), publisher: AIP Publishing.
 ing.
- ¹⁴Y. Liu, C. Paz-Soldan, L. Li, and Y. Sun, en"Role of 3D neoclassical particle flux in density
 ⁴⁵¹ pump-out during ELM control by RMP in DIII-D," Nuclear Fusion **60**, 036018 (2020).

- ⁴⁵² ¹⁵V. Rozhansky, P. Molchanov, E. Kaveeva, S. Voskoboynikov, A. Kirk, E. Nardon, D. Coster, and
- M. Tendler, en"Modelling of the edge plasma of MAST in the presence of resonant magnetic
 perturbations," Nuclear Fusion 51, 083009 (2011).
- ⁴⁵⁵ ¹⁶S. Mordijck, R. A. Moyer, and G. R. McKee, en"Changes in density fluctuations as a result
 ⁴⁵⁶ of resonant magnetic perturbations correlate with the density inverse scale length," Physics of
- ⁴⁵⁷ Plasmas **19**, 024504 (2012).
- ⁴⁵⁸ ¹⁷G. McKee, Z. Yan, C. Holland, R. Buttery, T. Evans, R. Moyer, S. Mordijck, R. Nazikian,
- T. Rhodes, O. Schmitz, and M. Wade, en"Increase of turbulence and transport with resonant
 magnetic perturbations in ELM-suppressed plasmas on DIII-D," Nuclear Fusion 53, 113011
 (2013).
- ⁴⁶² ¹⁸H. Müller, T. Lunt, W. Suttrop, T. Eich, R. Fischer, J. Fuchs, A. Herrmann, M. Kočan,
 ⁴⁶³ P. de Marné, and E. Wolfrum, en"Modification of scrape-off layer transport and turbulence by
 ⁴⁶⁴ non-axisymmetric magnetic perturbations in ASDEX Upgrade," Journal of Nuclear Materials
 ⁴⁶⁵ **438**, S64–S71 (2013).
- ¹⁹N. Vianello, C. Rea, M. Agostini, R. Cavazzana, G. Ciaccio, G. De Masi, E. Martines, A. Mazzi,
 B. Momo, G. Spizzo, P. Scarin, M. Spolaore, P. Zanca, M. Zuin, L. Carraro, P. Innocente,
 L. Marrelli, M. E. Puiatti, and D. Terranova, en"Magnetic perturbations as a viable tool for
 edge turbulence modification," Plasma Physics and Controlled Fusion 57, 014027 (2015).
- ⁴⁷⁰ ²⁰C. Rea, N. Vianello, M. Agostini, R. Cavazzana, G. De Masi, E. Martines, B. Momo, P. Scarin,
- 471 S. Spagnolo, G. Spizzo, M. Spolaore, and M. Zuin, en"Comparative studies of electrostatic tur-
- ⁴⁷² bulence induced transport in presence of resonant magnetic perturbations in RFX-mod," Nuclear
 ⁴⁷³ Fusion 55, 113021 (2015).
- ⁴⁷⁴ ²¹L. Cui, R. Nazikian, B. Grierson, E. Belli, T. Evans, N. Logan, D. Orlov, S. Smith, G. Staebler,
 ⁴⁷⁵ and P. Snyder, en"The energy confinement response of DIII-D plasmas to resonant magnetic
 ⁴⁷⁶ perturbations," Nuclear Fusion **57**, 116030 (2017).
- ⁴⁷⁷ ²²S. Liu, N. Yan, Y. Liang, H. Zhang, J. Xu, G. Xu, L. Wang, R. Chen, G. Hu, Y. Ye, Y. Sun, T. Shi,
- H. Wang, M. Wu, X. Wu, S. Gu, M. Jia, N. Chu, Q. Ma, Y. Wang, T. Zhang, X. Han, L. Chen,
- J. Liu, S. Xu, H. Wang, N. Zhao, W. Zhang, J. Qian, L. Zeng, L. Xu, S. Wang, H. Liu, Q. Zang,
- 480 Y. Yu, L. Liao, X. Gong, and EAST, en"Edge turbulence characteristics and transport during
- the ELM mitigation with n = 1 resonant magnetic perturbation on EAST," Nuclear Fusion 60,
- 482 082001 (2020).

- ²³M. W. Jakubowski, T. E. Evans, M. E. Fenstermacher, M. Groth, C. J. Lasnier, A. W. Leonard,
 O. Schmitz, J. G. Watkins, T. Eich, W. Fundamenski, R. A. Moyer, R. C. Wolf, L. B. Baylor, J. A. Boedo, K. H. Burrell, H. Frerichs, J. S. deGrassie, P. Gohil, I. Joseph, S. Mordijck,
 M. Lehnen, C. C. Petty, R. I. Pinsker, D. Reiter, T. L. Rhodes, U. Samm, M. J. Schaffer, P. B.
 Snyder, H. Stoschus, T. Osborne, B. Unterberg, E. Unterberg, and W. P. West, "Overview of the
 results on divertor heat loads in RMP controlled H-mode plasmas on DIII-D," Nuclear Fusion
 49, 095013 (2009), publisher: IOP Publishing.
- ⁴⁹⁰ ²⁴P. Snyder, K. Burrell, H. Wilson, M. Chu, M. Fenstermacher, A. Leonard, R. Moyer, T. Osborne,
- M. Umansky, W. West, and X. Xu, en"Stability and dynamics of the edge pedestal in the low
 collisionality regime: physics mechanisms for steady-state ELM-free operation," Nuclear Fusion
 47, 961–968 (2007).
- ⁴⁹⁴ ²⁵R. Nazikian, C. Paz-Soldan, J. \. e. Callen, J. \. e. deGrassie, D. Eldon, T. \. e. Evans, N. \. e.
- ⁴⁹⁵ Ferraro, B. \. e. Grierson, R. \. e. Groebner, S. \. e. Haskey, C. \. e. Hegna, J. \. e. King, N. \. e.
- Logan, G. \. e. McKee, R. \. e. Moyer, M. Okabayashi, D. \. e. Orlov, T. \. e. Osborne, J.-K.
- Park, T. \. e. Rhodes, M. \. e. Shafer, P. \. e. Snyder, W. \. e. Solomon, E. \. e. Strait, and
 M. \. e. Wade, "Pedestal Bifurcation and Resonant Field Penetration at the Threshold of EdgeLocalized Mode Suppression in the DIII-D Tokamak," Physical Review Letters 114 (2015),
 10.1103/physrevlett.114.105002, publisher: American Physical Society (APS).
- ²⁶C. Paz-Soldan, R. Nazikian, S. \. e. Haskey, N. \. e. Logan, E. \. e. Strait, N. \. e. Ferraro, J. \. e.
 Hanson, J. \. e. King, M. \. e. Lanctot, R. \. e. Moyer, M. Okabayashi, J.-K. Park, M. \. e.
 Shafer, and B. \. e. Tobias, "Observation of a Multimode Plasma Response and its Relationship to Density Pumpout and Edge-Localized Mode Suppression," Physical Review Letters 114
 (2015), 10.1103/physrevlett.114.105001, publisher: American Physical Society (APS).
- ⁵⁰⁶ ²⁷G. Lee, J. Kim, S. Hwang, C. Chang, H. Chang, M. Cho, B. Choi, K. Kim, K. Cho, S. Cho,
- 507 K. Choh, C. Choi, J. Choi, J. Choi, I. Choi, C. Do, T. Ha, J. Han, J. Hong, K. Hong, N. Hur,
- I. Hwang, K. Im, H. Jhang, Y. Jung, B. Kim, D. Kim, G. Kim, H. Kim, J. Kim, J. Kim, W. Kim,
- Y. Kim, K. Kwon, M. Kyum, B. Lee, D. Lee, H. Lee, J. Lee, S. Lee, H. Na, Y. Oh, J. Park,
- H. Ri, Y. Ryoo, K. Song, H. Yang, J. Yang, B. Yoo, S. Yoo, N. Yoon, S. Yoon, G. You, K. You,
- ⁵¹¹ W. Choe, D.-I. Choi, S. Jeong, D. Lee, Y. Bae, H. Kang, G. Kim, I. Ko, W. Namkung, J. Oh,
- Y. Bae, Y. Cho, B. Hong, G. Hong, C. Hwang, S. In, M. Ju, H. Lee, B. Oh, B. Yoon, S. Baang,
- H. Choi, J. Hwang, M. Kim, Y. Kim, S. Lee, J. Yee, C. Yoon, K.-H. Chung, S. Hong, Y. Hwang,
- 514 S. Kim, Y. Kim, K. Chung, J. Lim, D. Ha, S. Oh, K. Ryu, Q. Wang, T. Ko, J. Joo, S. Suh, C. Choi,

- J. Lee, Y. Lee, H. Shin, I. Song, J. Baek, I. Han, Y. Koh, P. Park, C. Ryu, J. Cho, D. Hwang,
- Y. Kim, J. Schmidt, H. Park, G. Neilson, W. Reiersen, R. Simmons, S. Bernabei, F. Dahlgren,
- L. Grisham, S. Jardin, C. Kessel, J. Manickam, S. Medley, N. Pomphrey, J. Sinnis, T. Brown,
- R. White, K. Young, J. Schultz, P. Wang, L. Sevier, M. Carter, P. Ryan, D. Swain, D. Hill,
- 519 W. Nevins, and B. Braams, en"The KSTAR project: An advanced steady state superconducting
- tokamak experiment," Nuclear Fusion 40, 575–582 (2000).
- ⁵²¹ ²⁸I. P. E. G. o. C. Transport, I. P. E. G. o. C. Database, and I. P. B. Editors, "Chapter 2: Plasma ⁵²² confinement and transport," Nuclear Fusion **39**, 2175–2249 (1999).
- ²⁹H. Zohm, en"On the Minimum Size of DEMO," Fusion Science and Technology 58, 613–624
 (2010).
- ³⁰F. Laggner, D. Eldon, A. Nelson, C. Paz-Soldan, A. Bortolon, T. Evans, M. Fenstermacher,
 B. Grierson, Q. Hu, D. Humphreys, A. Hyatt, R. Nazikian, O. Meneghini, P. Snyder, E. Unter-
- ⁵²⁷ berg, E. Kolemen, and t. DIII-D team, en"Real-time pedestal optimization and ELM control ⁵²⁸ with 3D fields and gas flows on DIII-D," Nuclear Fusion **60**, 076004 (2020).
- ³¹Y. In, J.-K. Park, Y. M. Jeon, J. Kim, G. Y. Park, J.-W. Ahn, A. Loarte, W. H. Ko, H. H. Lee, J. W.
 Yoo, J. W. Juhn, S. W. Yoon, and H. P. and, "Enhanced understanding of non-axisymmetric intrinsic and controlled field impacts in tokamaks," Nuclear Fusion 57, 116054 (2017), publisher:
 IOP Publishing.
- ³²C. Sung, G. Wang, T. L. Rhodes, S. P. Smith, T. H. Osborne, M. Ono, G. R. McKee, Z. Yan, R. J.
 Groebner, E. M. Davis, L. Zeng, W. A. Peebles, and T. E. Evans, en"Increased electron temperature turbulence during suppression of edge localized mode by resonant magnetic perturbations
 in the DIII-D tokamak," Physics of Plasmas 24, 112305 (2017).
- ³³M. Kim, J. Lee, W. H. Ko, S.-H. Hahn, Y. In, Y. M. Jeon, W. Suttrop, S. K. Kim, G. Y. Park, J.-W.
 Juhn, and J. H. Lee, en"Pedestal electron collisionality and toroidal rotation during ELM-crash
- suppression phase under n = 1 RMP in KSTAR," Physics of Plasmas 27, 112501 (2020).
- ⁵⁴⁰ ³⁴D. Eldon, E. Kolemen, J. Barton, A. Briesemeister, D. Humphreys, A. Leonard, R. Maingi,
- M. Makowski, A. McLean, A. Moser, and P. Stangeby, en"Controlling marginally detached
 divertor plasmas," Nuclear Fusion 57, 066039 (2017).
- ⁵⁴³ ³⁵T. Osborne, G. Jackson, Z. Yan, R. Maingi, D. Mansfield, B. Grierson, C. Chrobak, A. McLean,
- 544 S. Allen, D. Battaglia, A. Briesemeister, M. Fenstermacher, G. McKee, P. Snyder, and The
- 545 DIII-D Team, "Enhanced H-mode pedestals with lithium injection in DIII-D," Nuclear Fusion
- ⁵⁴⁶ **55**, 063018 (2015).

- ³⁶Q. Hu, R. Nazikian, B. Grierson, N. Logan, D. Orlov, C. Paz-Soldan, and Q. Yu, en"Wide Operational Windows of Edge-Localized Mode Suppression by Resonant Magnetic Perturbations
 ⁵⁴⁹ in the DIII-D Tokamak," Physical Review Letters **125**, 045001 (2020).
- ³⁷V. Rozhansky, E. Kaveeva, P. Molchanov, I. Veselova, S. Voskoboynikov, D. Coster, A. Kirk,
 S. Lisgo, and E. Nardon, en"Modification of the edge transport barrier by resonant magnetic
 perturbations," Nuclear Fusion 50, 034005 (2010).
- ⁵⁵³ ³⁸E. Viezzer, M. Cavedon, P. Cano-Megias, E. Fable, E. Wolfrum, D. J. Cruz-Zabala, P. David,
- R. Dux, R. Fischer, G. F. Harrer, F. M. Laggner, R. M. McDermott, U. Plank, T. Pütterich,
 M. Willensdorfer, and the ASDEX Upgrade Team, en"Dynamics of the pedestal transport during
- edge localized mode cycles at ASDEX Upgrade," Plasma Physics and Controlled Fusion 62,
 024009 (2020).
- ³⁹W. A. Houlberg, K. C. Shaing, S. P. Hirshman, and M. C. Zarnstorff, en"Bootstrap current
 and neoclassical transport in tokamaks of arbitrary collisionality and aspect ratio," Physics of
 Plasmas 4, 3230–3242 (1997).
- ⁴⁰S. Mordijck, R. Moyer, N. Ferraro, M. Wade, and T. Osborne, en"The radial electric field as
 a measure for field penetration of resonant magnetic perturbations," Nuclear Fusion 54, 082003
 (2014).
- ⁴¹J. Lee, Y. M. Jeon, Y. In, G. Y. Park, G. S. Yun, W. Lee, M. Kim, J. H. Lee, W. H. Ko, and H. K. P.
 and, "Direct evidence of E × B flow changes at the onset of resonant magnetic perturbationdriven edge-localized mode crash suppression," Nuclear Fusion **59**, 066033 (2019), publisher:
 IOP Publishing.
- ⁴²J.-K. Park, Y. Jeon, Y. In, J.-W. Ahn, R. Nazikian, G. Park, J. Kim, H. Lee, W. Ko, H.-S. Kim,
 N. C. Logan, Z. Wang, E. A. Feibush, J. E. Menard, and M. C. Zarnstroff, "3D field phasespace control in tokamak plasmas," Nature Physics 14, 1223–1228 (2018), publisher: Springer
 Science and Business Media LLC.
- ⁴³X. Chen, K. Burrell, T. Osborne, W. Solomon, K. Barada, A. Garofalo, R. Groebner, N. Luhmann, G. McKee, C. Muscatello, M. Ono, C. Petty, M. Porkolab, T. Rhodes, J. Rost, P. Snyder,
- G. Staebler, B. Tobias, Z. Yan, and the DIII-D Team, en"Stationary QH-mode plasmas with high and wide pedestal at low rotation on DIII-D," Nuclear Fusion **57**, 022007 (2017).
- ⁴⁴R. Nazikian, C. Petty, A. Bortolon, X. Chen, D. Eldon, T. Evans, B. Grierson, N. Ferraro,
- 577 S. Haskey, M. Knolker, C. Lasnier, N. Logan, R. Moyer, D. Orlov, T. Osborne, C. Paz-Soldan,
- 578 F. Turco, H. Wang, and D. Weisberg, en"Grassy-ELM regime with edge resonant magnetic

- perturbations in fully noninductive plasmas in the DIII-D tokamak," Nuclear Fusion 58, 106010
 (2018).
- ⁴⁵J. Lee, G. S. Yun, M. J. Choi, J.-M. Kwon, Y.-M. Jeon, W. Lee, N. C. Luhmann, and
 H. K. Park, "Nonlinear Interaction of Edge-Localized Modes and Turbulent Eddies in Toroidal
 Plasma undern=1Magnetic Perturbation," Physical Review Letters **117** (2016), 10.1103/physrevlett.117.075001, publisher: American Physical Society (APS).
- ⁴⁶H. Biglari, P. H. Diamond, and P. W. Terry, "Influence of sheared poloidal rotation on edge
 turbulence," Physics of Fluids B: Plasma Physics 2, 1–4 (1990), publisher: AIP Publishing.
- ⁴⁷T. S. Hahm and K. H. Burrell, "Flow shear induced fluctuation suppression in finite aspect ratio
- shaped tokamak plasma," Physics of Plasmas 2, 1648–1651 (1995), publisher: AIP Publishing.
- ⁴⁸R. Hager, C. S. Chang, N. M. Ferraro, and R. Nazikian, en"Gyrokinetic understanding of the
 edge pedestal transport driven by resonant magnetic perturbations in a realistic divertor geometry," Physics of Plasmas 27, 062301 (2020), arXiv: 2003.07130.
- ⁴⁹S. Taimourzadeh, L. Shi, Z. Lin, R. Nazikian, I. Holod, and D. Spong, en"Effects of RMP induced changes of radial electric fields on microturbulence in DIII-D pedestal top," Nuclear
 ⁵⁹⁴Fusion **59**, 046005 (2019).
- ⁵⁰S. Yang, J.-K. Park, N. Logan, C. Zhu, Q. Hu, Y. Jeon, Y. In, W. Ko, S. Kim, Y. Lee, and
 Y. Na, "Localizing resonant magnetic perturbations for edge localized mode control in KSTAR,"
 Nuclear Fusion 60, 096023 (2020).
- ⁵⁹⁸ ⁵¹Won-Ha Ko, Seungtae Oh, and Myeun Kwon, en"KSTAR Charge Exchange Spectroscopy Sys-⁵⁹⁹ tem," IEEE Transactions on Plasma Science **38**, 996–1000 (2010).
- ⁵²J. H. Lee, S. Oh, H. M. Wi, W. R. Lee, K. P. Kim, K. team, I. Yamada, K. Narihara, and
 K. Kawahata, en"Tangential Thomson scattering diagnostic for the KSTAR tokamak," Journal
 of Instrumentation 7, C02026–C02026 (2012).
- ⁶⁰³ ⁵³G. S. Yun, W. Lee, M. J. Choi, J. B. Kim, H. K. Park, C. W. Domier, B. Tobias, T. Liang,
- ⁶⁰⁴ X. Kong, N. C. Luhmann, and A. J. H. Donné, en"Development of KSTAR ECE imaging ⁶⁰⁵ system for measurement of temperature fluctuations and edge density fluctuations," Review of
- ⁶⁰⁶ Scientific Instruments **81**, 10D930 (2010).
- ⁶⁰⁷ ⁵⁴K. Lee, J.-W. Juhn, Y. Nam, Y. Kim, H. Wi, S. Kim, and Y.-c. Ghim, en"The design of two color
- ⁶⁰⁸ interferometer system for the 3-dimensional analysis of plasma density evolution on KSTAR,"
- ⁶⁰⁹ Fusion Engineering and Design **113**, 87–91 (2016).

- ⁵⁵L. Lao, H. St. John, R. Stambaugh, A. Kellman, and W. Pfeiffer, en"Reconstruction of current profile parameters and plasma shapes in tokamaks," Nuclear Fusion **25**, 1611–1622 (1985).
- ⁶¹² ⁵⁶A. Pankin, D. McCune, R. Andre, G. Bateman, and A. Kritz, en"The tokamak Monte Carlo fast
- ion module NUBEAM in the National Transport Code Collaboration library," Computer Physics
 Communications 159, 157–184 (2004).
- ⁵⁷J. Chung, J. Ko, J. Howard, C. Michael, G. v. Nessi, A. Thorman, and M. F. M. D. Bock,
 "Motional Stark effect diagnostics for KSTAR," Journal of the Korean Physical Society 65,
 1257–1260 (2014), publisher: Korean Physical Society.
- ⁵⁸O. Sauter, C. Angioni, and Y. R. Lin-Liu, en"Neoclassical conductivity and bootstrap current
- formulas for general axisymmetric equilibria and arbitrary collisionality regime," Physics of
- 620 Plasmas **6**, 2834–2839 (1999).
- ⁵⁹P. B. Snyder, R. J. Groebner, A. W. Leonard, T. H. Osborne, and H. R. Wilson, en"Development
 and validation of a predictive model for the pedestal height," Physics of Plasmas 16, 056118
 (2009).
- ⁶⁰H. Lütjens, A. Bondeson, and O. Sauter, "The CHEASE code for toroidal MHD equilibria," ⁶²⁵ Computer Physics Communications **97**, 219–260 (1996), publisher: Elsevier BV.
- ⁶¹A. Mikhailovskii, G. Huysmans, W. Kerner, and S. Sharapov, "Optimization of computational
 MHD normal-mode analysis for tokamaks," Plasma Physics Reports 23, 844–857 (1997).
- ⁶²⁸ ⁶²J.-k. Park, A. H. Boozer, and A. H. Glasser, en"Computation of three-dimensional tokamak and ⁶²⁹ spherical torus equilibria," Physics of Plasmas **14**, 052110 (2007).
- ⁶³G. S. Yun, W. Lee, M. J. Choi, J. Lee, M. Kim, J. Leem, Y. Nam, G. H. Choe, H. K. Park,
- H. Park, D. S. Woo, K. W. Kim, C. W. Domier, N. C. Luhmann, N. Ito, A. Mase, and S. G. Lee,
- ⁶³² "Quasi 3D ECE imaging system for study of MHD instabilities in KSTAR," Review of Scientific

Instruments **85**, 11D820 (2014), publisher: AIP Publishing.

- ⁶⁴Y. U. Nam, S. Zoletnik, M. Lampert, Kovácsik, and H. M. Wi, en"Edge electron density profiles
- and fluctuations measured by two-dimensional beam emission spectroscopy in the KSTAR,"
- Review of Scientific Instruments **85**, 11E434 (2014).
- ⁶⁵J. G. Bak, S. G. Lee, D. Son, and the KSTAR Project Team, en"Performance of the magnetic
 ⁶³⁸ sensor and the integrator for the KSTAR magnetic diagnostics," Review of Scientific Instruments
 ⁶³⁹ **75**, 4305–4307 (2004).
- ⁶⁶J. Candy, E. Belli, and R. Bravenec, en"A high-accuracy Eulerian gyrokinetic solver for colli-
- sional plasmas," Journal of Computational Physics **324**, 73–93 (2016).